

In situ Fracture Studies in Reactor Materials

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Outline

- Why the need for crack growth studies in materials for reactors in virgin and irradiated state
- Imaging facilities
 - What is tomography?
- Crack growth in virgin and irradiated graphite
 - In situ tomographic studies
 - Toughness determination
- Crack growth in toxic materials
- Conclusions
- Acknowledgements

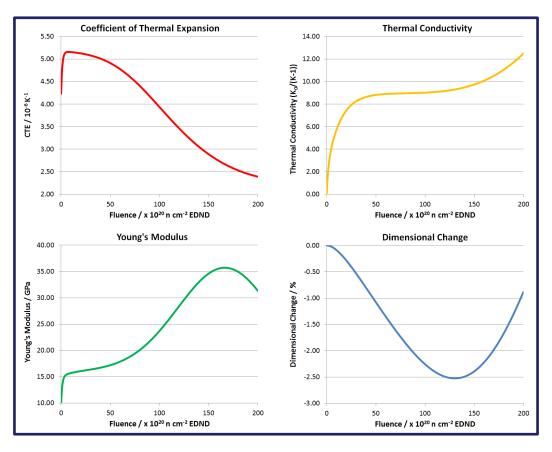


Why need for fracture studies?

- Mechanical behaviour and failure mechanisms are affected by irradiation
- Design rules, structural integrity calculations, and safety cases are often based on extrapolations from properties of virgin material
- It is necessary to assess the reliability of these extrapolation by (limited) experimentation
 - Doing fracture tests on large irradiated samples is a challenge...
 - Establishing procedures for safe transport, handling, testing, disposal of radioactive specimens is key



Property changes with irradiation



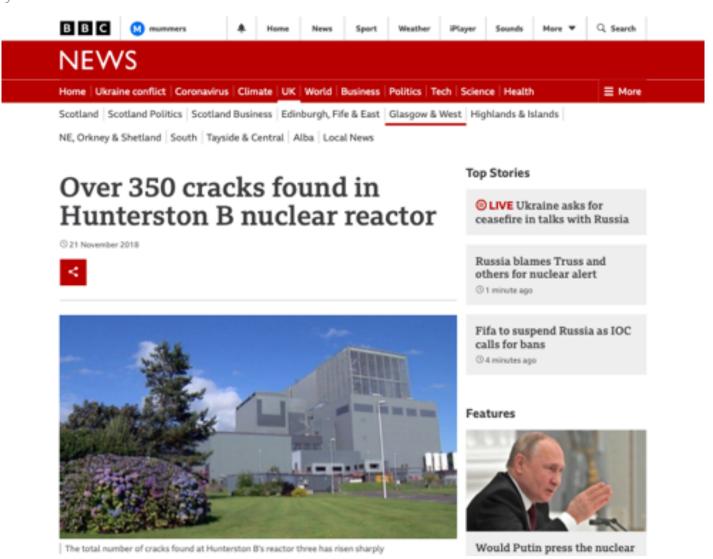
Prediction of behaviour in full scale components challenging



Why need for fracture studies?

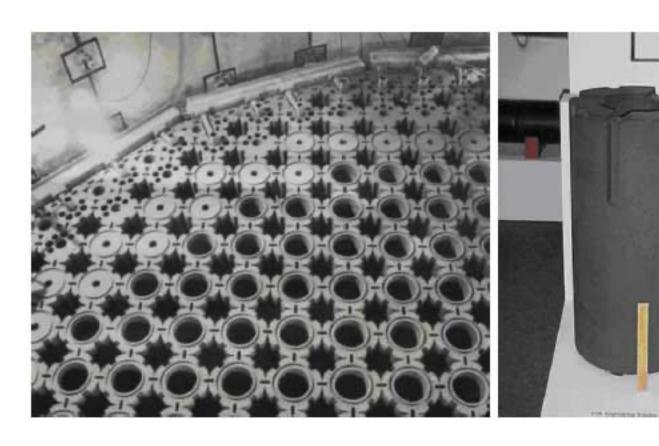
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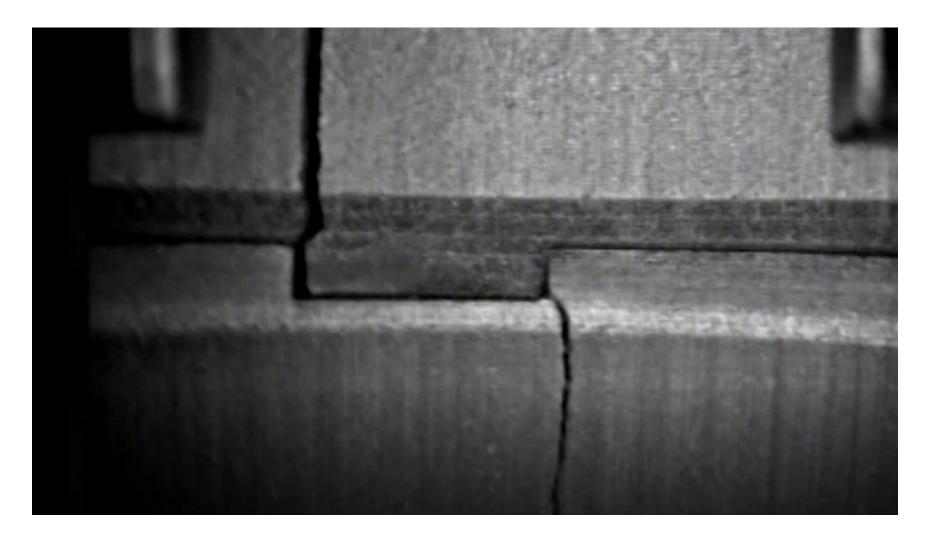
Is it safe to continue operation?





AGR core is a complex, interconnected structure of large faceted bricks containing sharp corners





How do cracks grow? Do they impair critically safe operation? Can we validate predictions of numerical models?



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X-Ray Tomography



What is X-ray Tomography?

- Transmitted intensity from a series of line projections of a cross section of the object at different angular orientations reconstructed to give 3-D map of x-ray absorption
- Advantages
 - Non-intrusive
 - Good spatial resolution (currently \approx 0.1 μ m in lab; \approx 5 nm at synchrotron sources)
 - Very sensitive to composition and density
 - Independent of specimen geometry
 - Can decouple μ and x



What do you get?



How can it be used?

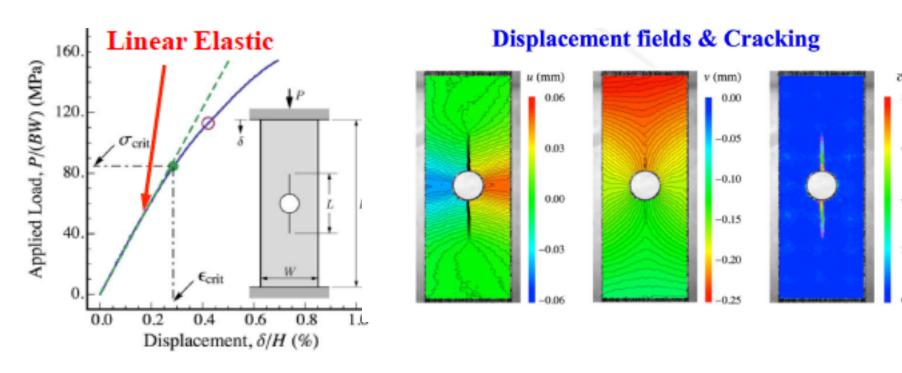
- Non-destructive 3D determination of structure
- In situ development of specimen (component, material) during environmental change
 - Loading, thermal, irradiation
 - Strain mapping through image correlation
- Basis for numerical model/digital engineering
 - Predictions based on actual structure
 - Solid mechanics; fluid dynamics; thermal transport



CRACK GROWTH IN VIRGIN AND IRRADIATED GRAPHITE USING SYNCHROTRON X-RAYS IN A NOVEL GEOMETRY



Test Geometry



- Easy to machine geometry
- Inherently stable crack propagation enables tomography
- Measurement of toughness properties and mechanisms

$$K_{IC} = \left(\frac{a_{\text{top}} + a_{\text{bottom}}}{2a}\right)^{1/2} \left\{\frac{1.1}{(1 + \frac{a_{\text{top}} + a_{\text{bottom}}}{2a})^{3.3}}\right\} \sigma \sqrt{\pi a}$$



Experimental Details

- Specimen geometry: 18 x 8 x 3 mm plates with a 2.8 mm hole in the centre.
- Loading condition: Uniaxial compression

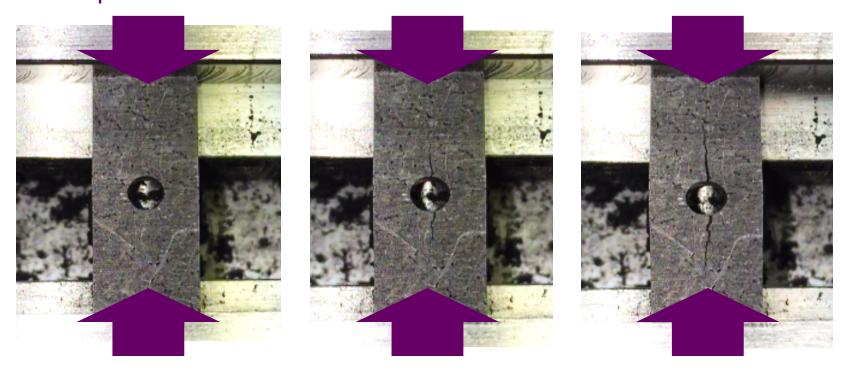
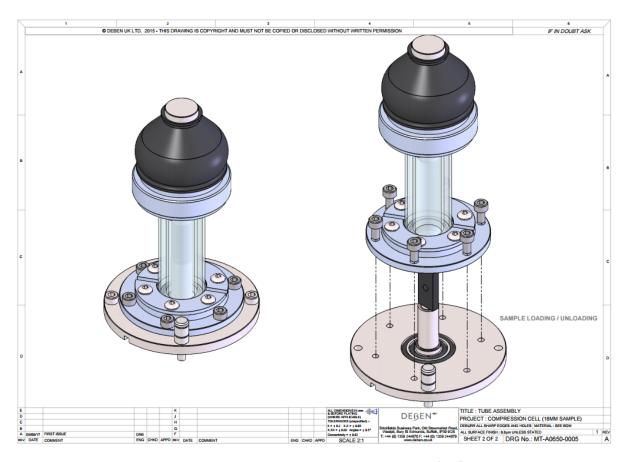


Fig. 4: The initiation and progressive growth of a crack around the hole in a plate of graphite in *in situ* SEM study



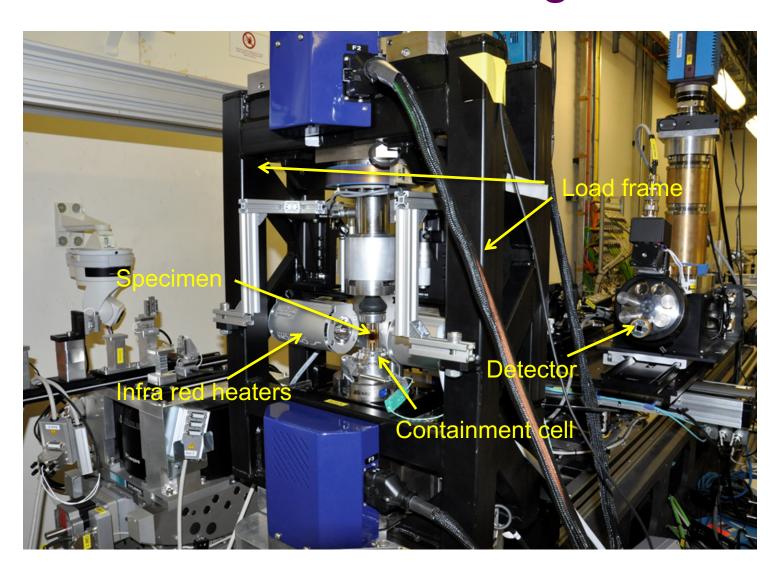
Double-Walled Containment Cell



Designed in partnership with H&S group at DLS World first in situ tomographic study of crack growth in irradiated graphite

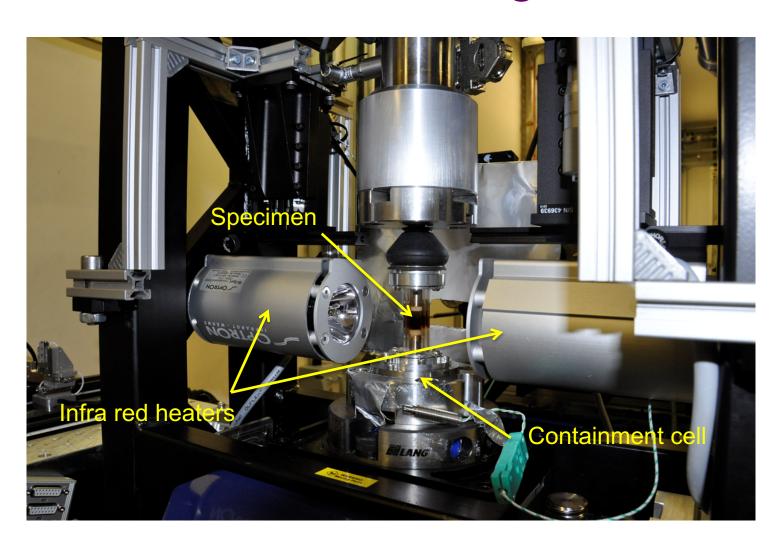


Experimental Setup: 113 Diamond Light Source





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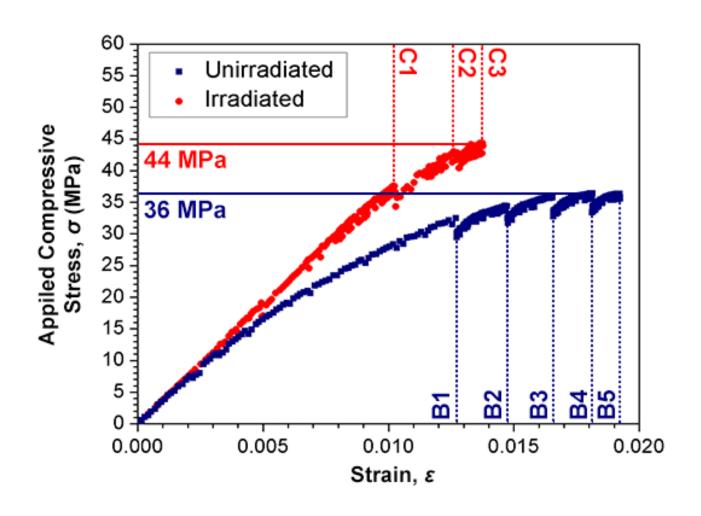


Experimental Details

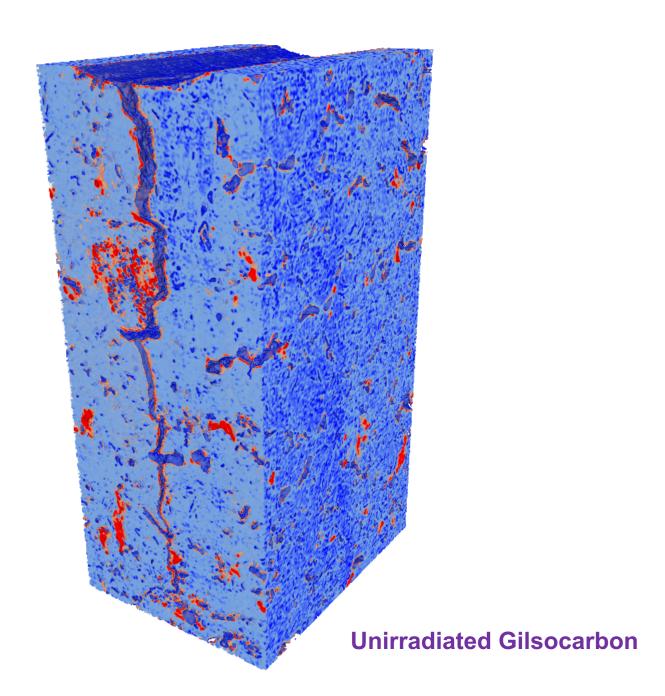
- Specimen geometry: 18 x 8 x 3 mm plates with a 2.8 mm hole in the Centre.
- H = 18mm, B = 8mm, W = 3mm, d = 2.8mm
- Loading condition: Uniaxial compression
- Materials:
 - Virgin Gilsocarbon (HPB)
 - Neutron Irradiated Gilsocarbon from HPB installed set
 - EDND 19.7 x 10²⁰ n cm⁻²; 4% weight loss
 - Machined from wings of WoF specimens
 - Measured bend strength 34.4MPa



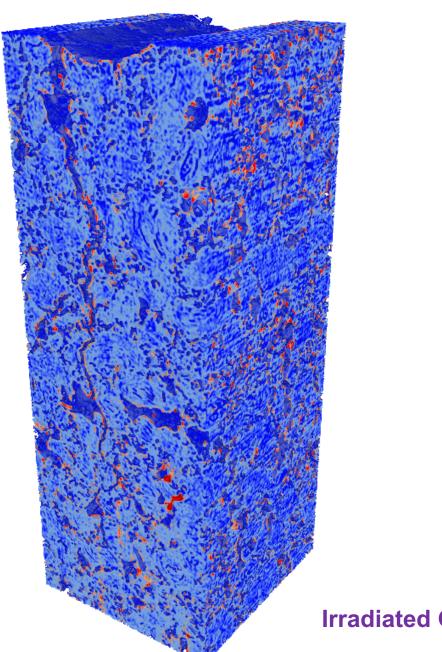
Mechanical behaviour







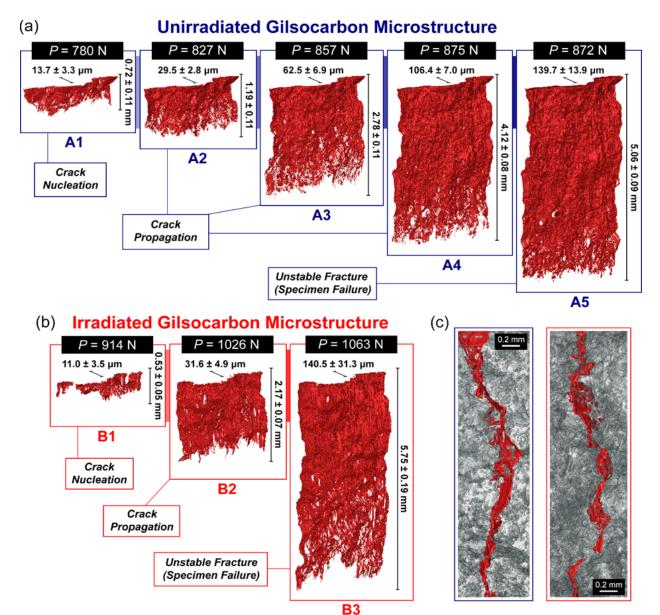




Irradiated Gilsocarbon

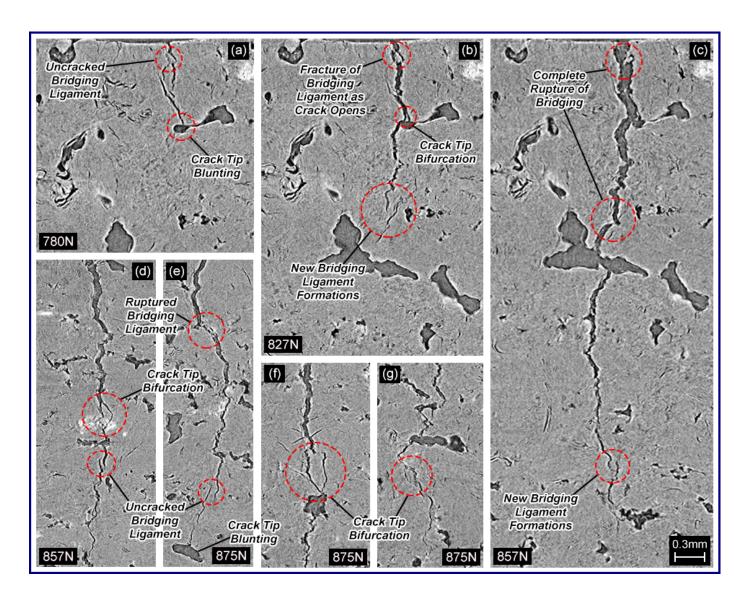


Crack geometries



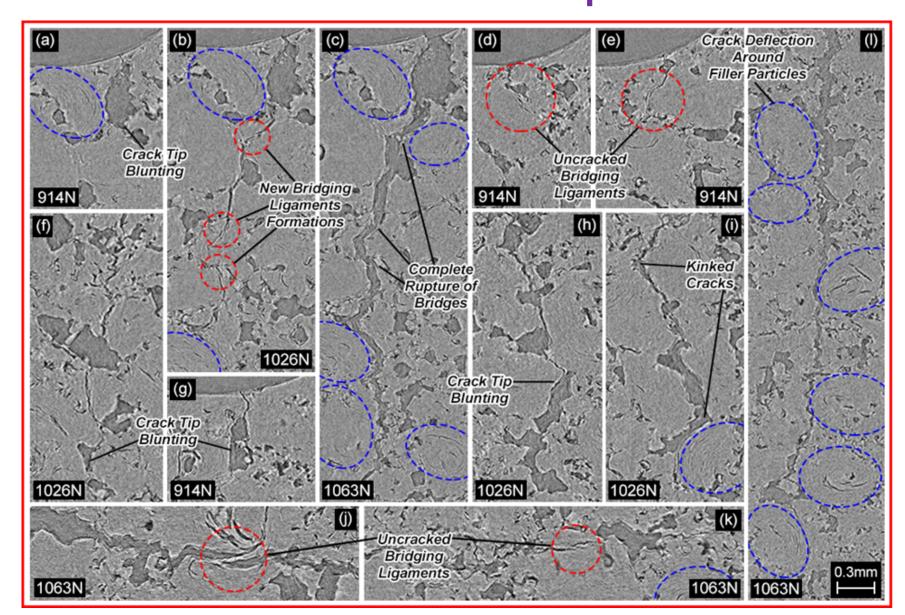


Virgin crack path



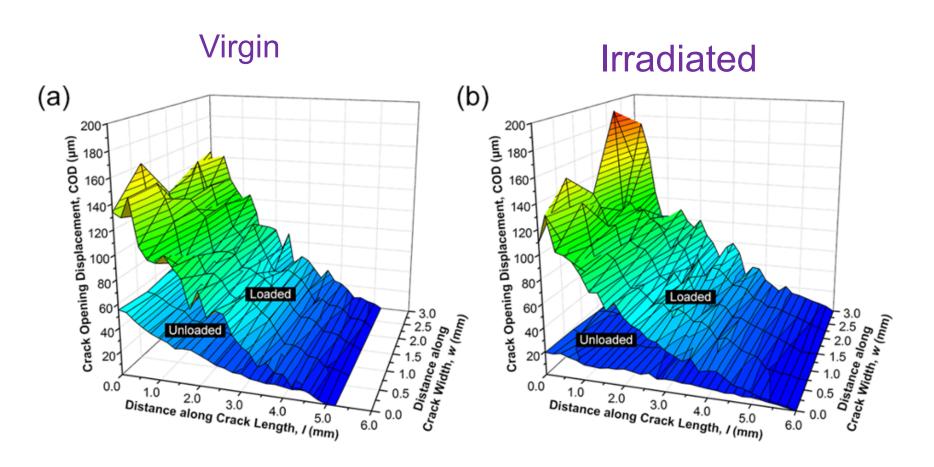


The University of Manchest radiated crack path





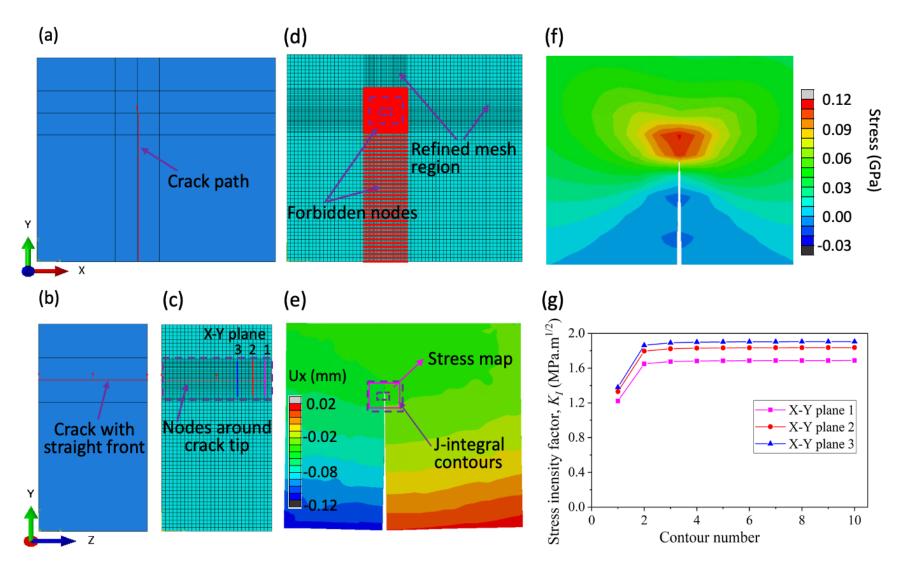
Crack mouth opening



On unloading, the virgin material displays significant residual opening, consistent with substantial plastic deformation. Conversely, the irradiated material recovers all deformation, consistent with elastic loading. This is a key indicator of reduced toughness of irradiated material



Toughness by J-integral





Pre-load

780 N

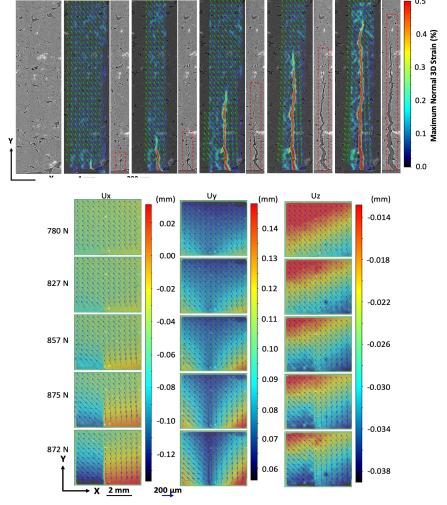
Crack paths and displacement fields

Virgin

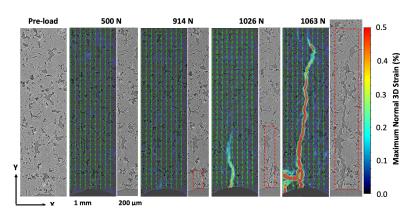
875 N

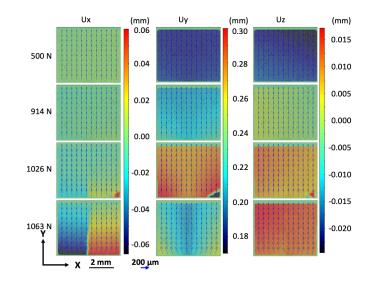
872 N

827 N



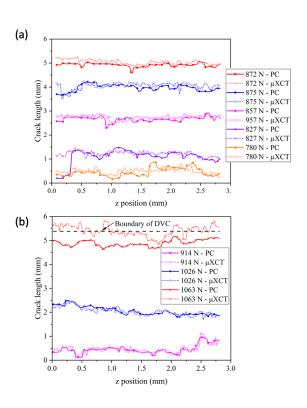
Irradiated





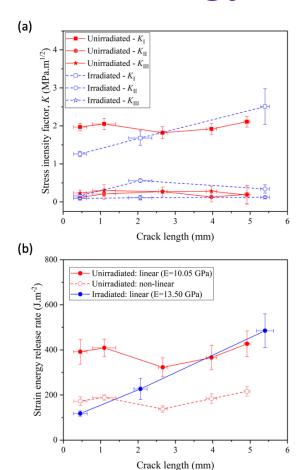


SIFs and fracture energy



Relatively straight crack front implies analysis robust

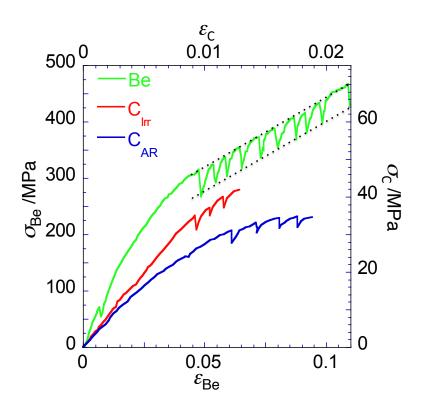


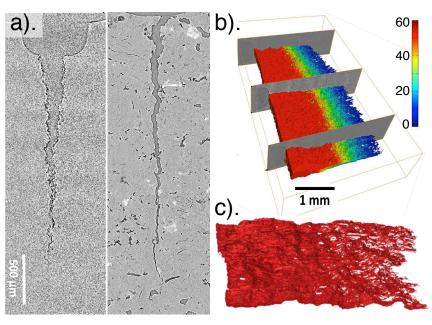


Both SIF and strain energy release rate reduced significantly on irradiation



Comparison of graphite and beryllium



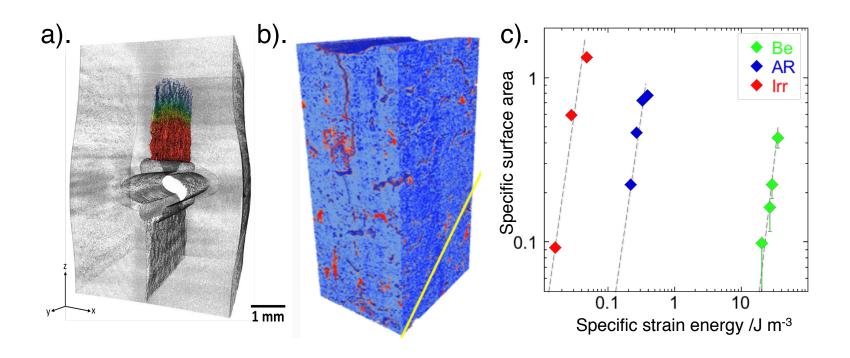


Stress/strain behaviour

Crack paths and profiles



Fracture energy

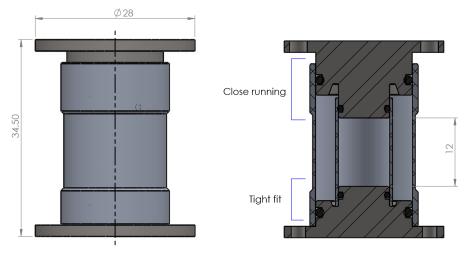


Crack profiles and surface areas

Effects of irradiation and fracture mechanism



New Containment Cell



Aluminium



Each containment layer is gas leak proof

84% of absorption @ 20 keV

Structural integrity remains at > 5 kN

Simpler to use but no ability for infra red heating

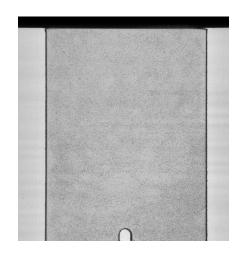
Simple manufacture so can be adapted to meet specimen and radiological needs

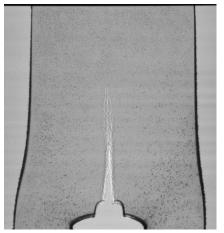


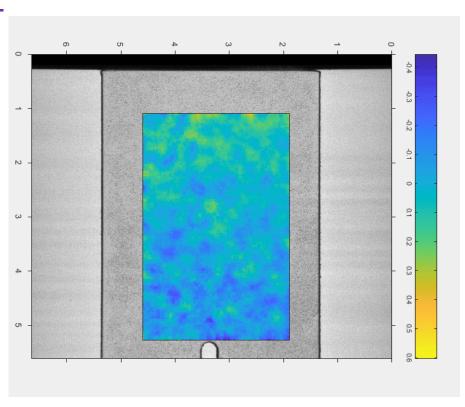
Beryllium: ITER first wall, toxic and lack of contrast

Contrast enhancement:

- Natural contrast : porosity, inclusions -
- > weak, work in progress
- Additional contrast : W powder







Surface displacement field (DIC):



Summary

- First in situ tomographic crack growth of irradiated graphite and beryllium
 - Compare effects of irradiation on propagation mechanisms
 - Measure strength and fracture energy
- Establishment of User Facility at DLS
 - Supports Active Handling Facility
 - Load frames and containment cells for radioactive and toxic materials
 - Experimental officers and Research Fellows



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